

Influence of Radiolytic Products on the Chemistry of Uranium VI in Brines

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Jean-Francois Lucchini, Donald Reed, Marian Borkowski, Andrzej Rafalski, James Conca,

Los Alamos National Laboratory, EES-12 Carlsbad Operations, Actinide Chemistry and Repository Science Program, + 1-505-234-5556, fax: +1-505-887-3051, lucchini@lanl.gov

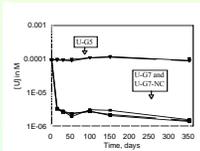
Introduction

Actinide chemistry in brines has been the subject of numerous investigations within the last two decades, because geological salt formations are being considered for radioactive waste disposal. Oxidation states, speciation and solubility of actinides in brine are a necessary step in the potential migration of actinides in the environment.

In the near-field chemistry of a salt repository of nuclear waste, ionizing radiations can strongly affect these processes, and thus the actinide mobility in concentrated saline solution. So the effects of radiolysis on high-saline brine under simulated repository conditions, and effect of the radiolytic by-products on actinide speciation are of particular importance.

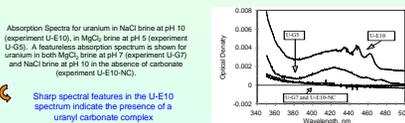
The aim of this work is to give a non-exhaustive review of the chemistry of Uranium VI under the possible effects of irradiated NaCl concentrated solutions in alkaline medium. We also introduce the experimental objectives planned to better understand the Uranium VI subsurface chemistry under the range of conditions expected in a nuclear waste repository.

Uranyl stability in brine

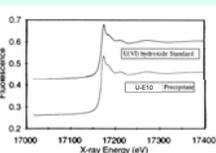


Concentration of uranium as a function of time in a MgCl₂ brine at pH 5 (experiment U-G6) and pH 7 (experiments U-G7 and U-G7-NC). Data for both unfiltered and 0.2 μm-filtered solutions are shown.

Steady-state uranium concentrations are obtained in the MgCl₂ brine, as well as in a NaCl brine (not shown in the figures)



Sharp spectral features in the U-E10 spectrum indicate the presence of a uranyl carbonate complex



XANES spectra of U(VI) hydroxide and the uranium precipitate recovered from the U-E10 experiment establish the precipitate as U(VI), rather than U(IV).

Uranium is stable in NaCl brine in the +6 oxidation state, as a carbonate or hydrolytic complex

* Reed D.T. (1997). Unpublished data. "Actinide Chemistry Stability/Speciation" Project. Argonne National Laboratory.

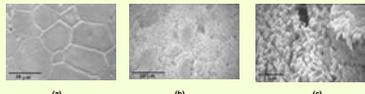
• Uranium(VI) chloride complexes are so weak, that they are not significant species in brine above pH 5.
 $UO_2Cl^+ + Cl^- \rightarrow UO_2Cl_2$ $\log K_{eq} = 0.17 \pm 0.02$
 $UO_2Cl_2 + Cl^- \rightarrow UO_2Cl_3$ $\log \beta^* = -1.1 \pm 0.4$

* Guehlin, F., J. Kari, G. Lemaire, R. Muller, A. Nguyen-Trung, C. Wanner H. (1989). "Chemical Speciation of Uranium in Aqueous Solutions." (Wanner, H. and Frenkel, I., eds.) OECD/NEA (1989)

• In synthetic NaCl-based brines containing Fe³⁺ (pH 8 to 13), no significant reduction of U(VI) was observed within 55 days. The lack of reduction of U(VI) in brine solutions could be attributed to the lower reducing capacity of Fe powder as brines than in dilute solutions.

* Ma Y., Bao L., Guo D., Fan Y. (2003). "Determining the Distribution of Fe³⁺ in NaCl and Synthetic Brine Solutions." www.lanl.gov

Effect of H₂O₂ on uranyl in water: formation of uranyl peroxide



Scanning electron microscopy (SEM) images of (a) unetched UO₂ surface and (b) and (c) etched UO₂ surface in water after 27 h of gamma beam irradiation.

Increase of the specific surface of the material and its solubility

- Tetrahydrate uranium peroxide (synthesized at room temperature) is reported to have a solubility product of $\log K_{sp} = -2.826$ for the reaction: $UO_2^{2+} + H_2O_2 = UO_2(OH)_2 + 2H^+$ in aqueous solution of a nearly neutral pH value.
- The two different polymorphs, which appear in nature as stutite (UO₂(OH)₂) and meta-stutite (UO₂(OH)₂), were found to have solubilities (measured in pH area close to 7) of 10^{-10} and $1.5 \cdot 10^{-10}$ M respectively.
- UO₂ solubility is about 10^{-10} M in the same conditions.

* Anne M. (2002). "Kinetics of the water radiolysis product H₂O₂ with the dissolution of nuclear fuel in natural ground water and dissolved state." www.lanl.gov
 * Lucchini, J.F. (2001). "Effect of Water Alpha Radiolysis on the Spent Nuclear Fuel (SNF) Matrix." Thesis, University of Paris XI Orsay, France

Uranyl peroxide properties

- Stutite can form from schoepite in low peroxide concentrations: $1.1 \cdot 10^{-14}$ M H₂O₂ (calculations).
- Stutite is the stable phase relative to dehydrated schoepite when hydrogen peroxide occurs.
- Stutite is thermodynamically unstable in systems with no peroxide, and at room temperature.

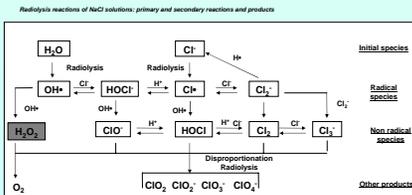
Solution composition have an impact on precipitation of uranyl peroxide.

- The formation of uranyl peroxide is inhibited in high chloride concentration solutions (> 2.7M).
- Even at initial H₂O₂ concentration of 10⁻⁶M, no uranyl peroxide forms if there is a scavenger of H₂O₂ in solution.
- However, if the peroxide concentration is high enough, the scavenging mechanism should be overruled and formation of UO₂ should take place, as in the case of natural formation of stutite minerals.
- Dissolution of uranyl peroxide occurs at low pH, or in presence of a strong oxidant (such as OCl⁻).

* Anne M. (2002). "Kinetics of the water radiolysis product H₂O₂ with the dissolution of nuclear fuel in natural ground water and dissolved state." www.lanl.gov
 * Brown S.A. (1981). "Thermodynamic properties of minerals and related substances." www.lanl.gov
 * Muller, C. (1989). "Quaternary progress of Uranium: transition on millerite." www.lanl.gov

Influence of chloride concentrations on the radiolysis mechanism

At low solute concentrations, the solute does not greatly affect the molecular yield of radiolytic products in water. However, species that react with OH• or act as scavengers (such as Cl⁻ or Br⁻) gradually lower the molecular yield of H₂O₂ as the concentration of the scavengers is increasing.



The overall radiolytic production of H₂O₂ should be low in brines.

* Reppelmann, K., Ma, J., Lemaire, G. (1986). "The radiolysis behavior of Fe³⁺ in saline solutions under radiolytic attack." www.lanl.gov

Hypochlorite formation in NaCl solutions

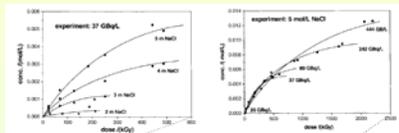


Fig. 1. Hypochlorite formed radiolytically in solution of various NaCl concentrations at a constant alpha activity concentration of 27 Ci/hγ.

The formation of hypochlorite ion increases with the chloride concentration and the dose.

- Calculations show that the formation of hydrogen peroxide decreases when the chloride concentration increases in the solution.

* Ma Y., Fan Y., Guo D., Sun T. (1998). "Spectroscopic investigation on the formation of hypochlorite by alpha radiolysis in concentrated NaCl solutions." www.lanl.gov

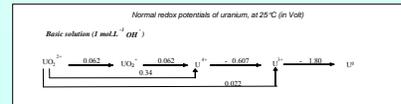
Radiolytic reactions in brines can cause...

- ...the change in pH. The initial effect of radiolysis of neutral saline solutions is a rapid decrease in pH locally to 4, followed by a gradual increase over a period of days to neutrality.
- ...an increase of the redox potential (Eh) of the solution. The redox potential may reach about +1300 mV in acidic solutions. The solutions are then strongly oxidizing.

Species	λ_{max} (nm)	ϵ (L mol ⁻¹ cm ⁻¹)	Redox potential (V)
Radicals			
OH•	—	—	E°(•OH/Cl ⁻) = +2.41
Cl ₂ •	340	8800	E°(Cl ₂ •/Cl ⁻) = +2.09
HOCl•	358	3760	E°(HOCl•/Cl ⁻) = +1.69
ClO•	280	890	E°(ClO•/Cl ⁻) = +1.82 (estimated)
Non radicals			
Cl ₂	333	65	E°(Cl ₂ /Cl ⁻) = +1.49
Cl ₃	325	196	E°(Cl ₃ /Cl ⁻) = +1.36 (estimated)
HClO	235	105	E°(HClO/Cl ⁻) = +1.48
ClO	292	350	E°(ClO/Cl ⁻) = +1.69
ClO ₂	260	160	E°(ClO ₂ /Cl ⁻) = +1.66
ClO ₃	360	3000	E°(ClO ₃ /Cl ⁻) = +1.31

* Reppelmann, K., Ma, J., Lemaire, G. (1986). "The radiolysis behavior of Fe³⁺ in saline solutions under radiolytic attack." www.lanl.gov

Potential radiolysis effects on uranium chemistry in brines: no redox change...



Brine radiolysis should maintain uranium in solution as U(VI).

... but hypochlorite ion insertion into uranyl complexes ?

- In solubility studies of schoepite (UO₂)(OH)₂•H₂O with hypochlorite ion in 0.1M NaCl at 25°C, under CO₂-free atmosphere, XRD patterns for the residual precipitates showed the introduction of hypochlorite ion in precipitates.
- Hypochlorite ion could displace hydroxide in (UO₂)(OH)₂, a predominant species in solution, to form a mixed complex and could also be incorporated into uranium precipitates. This would affect the overall uranium solubility.

* Ma Y., Bao L., Guo D., Sun T. (1998). "Effects of hypochlorite ion on the stability of schoepite at 25°C in neutral saline solutions." www.lanl.gov

Effect of hypochlorite ion on schoepite

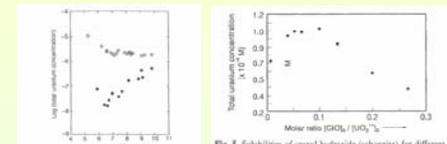


Fig. 1. Solubility of uranyl hydroxide (schoepite) for different molar ratios of hypochlorite ion concentration to initial uranyl ion concentration in 0.1 M NaCl solution at pH 8.3, and

The presence of hypochlorite ion enhanced the solubility of schoepite 10 to 100 times in the range of pH 6.0-9.0, compared with its solubility in the absence of hypochlorite ion. This effect is reduced when the molar ratio [OCl⁻]/[UO₂²⁺] increases.

* Ma Y., Bao L., Guo D., Sun T. (1998). "Effects of hypochlorite ion on the stability of schoepite at 25°C in neutral saline solutions." www.lanl.gov

Conclusion

This poster gives an overview of the literature about the chemistry of uranium(VI) in brines in the presence of radiolytic products.

Major statements that can be made are:

- Some data exist on the effects of radiolysis on NaCl brines, but these are incomplete and do not fully define radiolytic effects in more realistic brines.
- U(VI) is not expected to be reduced in irradiated brines as a consequence of radiolysis. Non-radiolytic redox pathways do exist and are currently under investigation, e.g., bio and iron.
- Radiolytic products (H₂O₂, OCl⁻), as well as brine and repository components (Fe, Br, etc.), affect the speciation of uranium(VI) in brines.
- Issues related to precipitation/complexation effects on U(VI) speciation are being addressed experimentally.

Experimental objectives

Brine radiolysis presents a complex mechanism. The interaction of uranium with radiolytic species in solution remains a matter of investigation. Our study will focus on the potential effects of radiolytic molecular products H₂O₂ and OCl⁻ on the uranium speciation in brines.

The overall reducing conditions of a repository (metals from containers, bioactivity) can mitigate the oxidizing effects of radiolysis and initial radiolytic products. The redox stability of U(VI) in the presence of key repository components (e.g. iron) in brines will be addressed in our experimental studies.

Publications concerning the solubility of uranium are numerous in the literature, but very few were done in brine and under conditions that represent geological salt repository conditions. The effective solubility of uranium(VI) as a function of key subsurface parameters will be established (e.g., brine composition, carbonate concentration, pH, and redox).